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REGARDING THE REPORT ON IMPLEMENTATION OF THE POST-PROJECT ANALYSIS PROGRAMME FOR THE NEW INSTALLATIONS OF IGNALINA NUCLEAR POWER PLANT FOR THE YEAR 2025

The Ministry of Environment of the Republic of Lithuania presents its compliments to the Environment State Bureau of the Republic of Latvia and the General Directorate for Environmental Protection of Poland and hereby provides the Transboundary Impact Monitoring Report for the New Installations of Ignalina Nuclear Power Plant (NPP) for the year 2025 (hereinafter referred to as the Report). The objective of the Report is to provide an overview of the results of annual environmental monitoring (radiological and chemical) in accordance with the Transboundary Impact Monitoring Programme for the new Ignalina NPP nuclear installations:

- Dry Type Interim Spent Nuclear Fuel Storage Facility (ISFSF, B1 Project);
- Solid Waste Management and Storage Facilities (SWMSF, B2/3/4 Project) consisting of Solid Waste Retrieval Facility (SWRF, B2 Project) and Solid Waste Treatment and Storage Facility (SWTSF, B3/4 Project);
- Landfill Disposal Facility for Short-Lived Very Low-Level Waste (Landfill Repository, B19 Project).

We sincerely hope that you will find this information useful, and we would like to take this opportunity to express our appreciation for the fruitful cooperation between our institutions in the field of environmental protection.

Vice-minister Giedrė Ričkutė

ENCLOSED: Transboundary Impact Monitoring Report for year 2025. New INPP Nuclear Installations: Interim Spent Fuel Storage Facility (B1 Project), Solid Radioactive Waste Retrieval Facility (B2-1 and B2-2 Projects), Solid Radioactive Waste Treatment and Storage Facility (B3/4 Project) and Landfill Disposal Modules (B19-2 Project), 37 pages.

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Altra

Ignalina Nuclear Power Plant

TRANSBOUNDARY IMPACT MONITORING REPORT FOR YEAR 2025.

NEW IGNALINA NPP NUCLEAR FACILITIES: INTERIM SPENT FUEL STORAGE FACILITY (B1 PROJECT), SOLID RADIOACTIVE WASTE RETRIEVAL FACILITY (B2-1 AND B2-2 PROJECTS), SOLID RADIOACTIVE WASTE TREATMENT AND STORAGE FACILITY (B3/4 PROJECT) AND LANDFILL DISPOSAL MODULES (B19-2 PROJECT)



2026

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1 INTRODUCTION

The purpose of the present document is to provide an overview of the annual environmental monitoring (radiological and chemical) summarising results within the scope of implementation of the Transboundary Impact Monitoring Programme for the new Ignalina NPP nuclear facilities for which transboundary environmental impact assessment has been carried out pursuant to the Convention on Environmental Impact Assessment in a Transboundary Context of the United Nations Economic Commission for Europe (Espoo, 1991), including the results of the cumulative impact to the population and the environment as a result of operation of all the nuclear facilities located on the INPP site.

The present Report includes an overview of the environmental impact of the following INPP spent nuclear fuel and radioactive waste management and storage facilities:

- Dry Type Interim Spent Nuclear Fuel Storage Facility (ISFSF, B1 Project);
- Solid Waste Management and Storage Facilities (SWMSF, B2/3/4 Project) consisting of Solid Waste Retrieval Facility (SWRF, B2 Project) and Solid Waste Treatment and Storage Facility (SWTSF, B3/4 Project);
- Landfill Disposal Facility for Short-Lived Very Low-Level Waste (Landfill Repository, B19 Project).

* Transboundary Impact Monitoring Programme of the project on reconstruction and transformation of the Ignalina NPP Bituminized Radioactive Waste Storage Facility (Building 158) into the Near Surface Repository (B20 Project) will be developed separately or by supplementing the Post-Project Analysis Programme for the New Ignalina NPP Nuclear Facilities [1] (preliminary in 2027) after approval by the regulatory body of the design documents and safety analysis report, including environmental monitoring programme, and it will be presented to the interested parties.

The purpose of the continuous and systematic environmental monitoring carried out at the INPP pursuant to the legal acts of the Republic of Lithuania [2-7] is to:

- limit radionuclide pathways into the environmental components, thus protecting the INPP personnel, the population, and the environment against hazardous radionuclide impact, as well as to forecast contamination effects;
- prove conformity with the discharge limits set for airborne and waterborne discharges;
- demonstrate that the exposure doses of the personnel and the population do not exceed the limit dose values set in the normative documents;
- analyse and assess efficiency of the implemented environmental protection measures;
- accumulate, analyse and submit to the state institutions and the population the information on the environment condition on the INPP territory and the sanitary protection zone, including the interested neighbouring countries in implementing the provisions of the Espoo Convention.

The environmental monitoring of the Ignalina NPP region is performed in accordance with the approved Environmental Monitoring Programmes [8-10] developed on the basis of the above-mentioned legal acts of the Republic of Lithuania.

1.1 Abbreviations and Definitions

BOD	Biochemical Oxygen Demand;
COC	Chemical Oxygen Consumption;
INPP	Ignalina Nuclear Power Plant;
ISFSF	Interim Spent Fuel Storage Facility;
MPC	Maximum permissible concentrations;
NF	Nuclear facility;

RU	Retrieval Unit;
RW	Radioactive waste;
SNF	Spent nuclear fuel;
SPZ	Sanitary Protection Zone;
SZ	Surveillance Zone;
SSC	Structures, system and components important to safety;
SWRF	Solid Radioactive Waste Retrieval Facility;
SWMSF	Solid Radioactive Waste Management and Storage Facility;
SWTSF	Solid Radioactive Waste Treatment and Storage Facility.

Representative person is assumed as a representative person, a member of a representative public group who is subject to significant exposure due to airborne and waterborne discharges and who lives close to the INPP SPZ. The way of life, eating habits and activities of a representative person are characteristic of farmers, fishers and gardeners. The exposure dose of a representative person is evaluated taking into consideration all exposure pathways from the radiation source.

For calculation of the exposure doses of a representative person the exposure dose calculation methodology presented in Generic Models for Use in Assessing the Impact of Discharges of Radioactive Substances to the Environment [11] was applied which uses conservative radionuclide dispersion models and conservative general parameter values in the models.

The Lithuanian Hygiene Standard HN 73:2018 “Basic Standards of Radiation Protection” [3] establishes the dose constraint of annual effective dose - 0.2 mSv for the public due to exposure resulting from airborne and waterborne discharges from the nuclear facility and the exposure resulting directly from the NF. Exposure of the public is evaluated based on the exposure dose calculated for a representative person.

1.2 Interim Spent Nuclear Fuel Storage Facility, ISFSF (B1 Project)

The ISFSF is located within approximately 0.5 km to the south of the INPP Units. The territory of the ISFSF occupies 5.93 ha. The ISFSF site is connected to the INPP via railway as well as the road connection. The transfer of the casks to the ISFSF was performed by the rail transport. In total 190 casks are temporary stored at the ISFSF, 22 casks out of the total number of casks contain damaged and experimental fuel bundles.

Three basic areas are distinguished at the main ISFSF storage building: Reception Hall, Storage Hall, and Fuel Inspection Hot Cell. The building and all the structures were designed taking into consideration the seismic loads and explosion impact. The structures of the ISFSF building are designed in such a way as to protect the objects inside the building against consequences of external meteorological and seismic phenomena, as well as human activity (e.g., external fire, transportation accidents, etc.). The building stability calculations were performed in compliance with the requirements of valid normative documents considering the initial impact data of the seismic loads and explosion wave. For more detailed information regarding the seismic safety and measures taken to ensure it for all the new NF, as well as the technical surveillance measures of the new NF building structures, see the Post-Project Analysis Programme for the New Ignalina NPP Nuclear Facilities [1]. Commissioning of the ISFSF and start-up of the commercial operation was performed following the regulatory requirements of the Republic of Lithuania.

CONSTOR® RBMK1500/M2 casks are designed for storage period of not less than 50 years from the scheduled completion of transfer of all SNF into the ISFSF without any need for scheduled intervention during the entire storage period plus an allowance of 5 years for the subsequent transport of the fuel to the final storage site. The storage configuration of the cask

CONSTOR®RBMK1500/M2 loaded with 91 Spent Fuel Assembles or 182 Fuel Bundles is 102 Fuel Bundles in the 32M Basket and 80 Fuel Bundles in the Ring Basket.

Loading of the ISFSF with the SNF casks, including fresh fuel assemblies transportation from the on-site Fresh Nuclear Fuel Store (Bld. 165) for interim storage, was completed on 30 December 2022.

In total 190 CONSTOR®RBMK1500/M2 type casks containing 15555 of spent fuel assemblies, 1 cartridge with the fuel debris collected and 8 transport packaging containers containing 75 fresh fuel assemblies and 4 packaging cans with fresh fuel pellets are stored at the ISFSF.



Figure 1.2-1 ISFSF building

Information related to the Operation of the ISFSF

The ISFSF is operated strictly following the requirements of the Interim Spent Nuclear Fuel Storage Facility Operation Technical Specification [12] which was last updated in December 2022 to include requirements related to fresh fuel storage at the ISFSF. Safety of the ISFSF is ensured by the safe operation of the ISFSF process equipment for handling of storage casks and the building infrastructure systems, their timely technical maintenance, monitoring, periodic testing and inspections.

No breach of the normal operation conditions and operational limits were identified during the ISFSF operation in 2025. All casks with the spent nuclear fuel are stored at their pre-designed storage locations. All systems and components of the storage facility were operated without failures. No changes to the safety features of the ISFSF site have been identified. No initiating events of either natural or man induced origin as provided for in the ISFSF safety justification documents have been identified.

Mandatory monitoring of the surface temperature of each cask body is performed once per month. The measured temperature of all the casks was below the established limit and is dependent on the outside temperature seasonal fluctuations.

Continuous monitoring of the radiological parameters at the ISFSF is performed in compliance with the annual ISFSF radiological monitoring schedule.

All the procedures related to the ISFSF operation are carried out in compliance with the Nuclear Safety Requirements BSR-3.1.1-2016 “Spent Nuclear Fuel Handling at the Dry Type Spent Nuclear Fuel Storage Facility” [13].



Figure 1.2-2 ISFSF casks storage hall

1.3 Solid Radioactive Waste Management and Storage Facility, SWMSF (B2/3/4 Project)

The Solid Radioactive Waste Management and Storage Facility (SWMSF, B2/3/4 Project) consists of separate facilities located on two different sites:

- Solid Waste Retrieval Facility (SWRF, B2 Project) constructed within the boundary of the INPP site close to the existing Interim Radioactive Waste Storage Facilities (Bld. 155, 155/1, 157 and 157/1);
- Solid Waste Treatment and Storage Facility (SWTSF, B3/4 Project) constructed on a separate site located approximately 0.6 km to the south from the Ignalina NPP adjacent to the Interim Spent Fuel Storage Facility.

The main objectives of the SWMSF are as follows:

- Waste retrieval from the existing storage buildings 155, 155/1, 157 and 157/1;
- Sorting, preliminary treatment and packaging of waste retrieved from the storage buildings;
- Waste processing (fragmentation, incineration, compaction);
- Loading of processed and dedicated for interim storage waste into appropriate containers subject to the generated waste class;
- 50-years interim storage of RW packages.

1.4 Solid Radioactive Waste Retrieval Facility, SWRF (Retrieval Unit 1, B2-1 Project and Retrieval Units 2,3, B2-2 Project)

The purpose of the SWRF is to retrieve solid radioactive waste from its present storage location, i.e. Interim Radioactive Waste Storage Facilities (Bld. 155, 155/1, 157 and 157/1), pre-sort and package this waste. The SWRF consists of three Retrieval Units (RU), the Landfill Separation Facility and the Control Building. More detailed information on the description of SWRF, retrieved waste and the operations performed therein is provided in the Post-Project Analysis Programme for the New INPP Nuclear Installations [1].

The Retrieval Unit 1 (RU1) is used to retrieve RW from buildings 155 and 155/1, to receive Group 1 waste retrieved from buildings 157 and 157/1 (RU2), to receive class A operational RW, to perform pre-sorting, processing and radiological characterization of the waste; to transfer pre-sorted RW either to the Landfill Separation Facility sorting area or to SWTF for further treatment.

Waste is retrieved by using remotely operated vehicles which enter the waste storage compartments of buildings 155 and 155/1 via access apertures cut in the side of the waste buildings. The remote control of all operations is carried out from the control room arranged in the control building.

The Retrieval Unit 2 (RU2) is used to retrieve, pre-sort and pack Group 1 and Group 2 RW from buildings 157 and 157/1. RU2 is a mobile unit located on the top of the waste storage building. Waste retrieval is performed from the top of the waste storage compartments, after removal of closure panels. Waste is retrieved remotely with a girder crane equipped with specific grabs, which is lowered through an existing aperture of the roof slab of the waste compartment. Then the waste is loaded into appropriate waste transport containers for transfer to corresponding waste sorting and processing sites.

The Retrieval Unit 3 (RU3) is used to retrieve Group 3 RW from compartments 1 and 4 of building 157. Like the RU2, the RU3 is a mobile framed metal structure located on the top of the storage building. The waste retrieval is performed from the top of the compartments of building 157 after removing plugs from waste compartment roofs. All waste retrieval and packaging operations are performed inside the Unit with a remote tool carrier arm (manipulator). Operations are remotely controlled from the Control Building.



Figure 1.4-1 Retrieval Unit 1



Figure 1.4-2 Retrieval Units 2 and 3

Information related to the Operation of the Retrieval Units 1, 2, 3 (B2-1, B2-2 Projects)

The following quantities of RW of class A (very low-level waste, short-lived) was retrieved from Bld. 155:

- 260 m^3 during 2025;
- In total 494 m^3 of RW was retrieved from the beginning of the waste retrieval process from Bld. 155.

The following quantities of RW of class A was retrieved from Bld. 155/1:

- No RW was retrieved in 2025;
- In total 2009.7 m^3 of RW was retrieved from the beginning of the waste retrieval process from Bld. 155/1.

In total 2503.7 m^3 of RW of class A was retrieved from the beginning of the waste retrieval process from both Buildings 155/1 and 155.

Dismantling waste originating from Bld. 101/1,2, 130/2, 159B and transported to RU1, as well as industrial waste of class A retrieved from Bld. 157/1 separate compartments was also treated at the Landfill Separation Facility of RU1.

2004 m^3 of RW of classes A, B, C (very low, low and intermediate level waste, short-lived) was retrieved from Bld. 157/1 during 2025 and in total 5743.6 m^3 of RW was retrieved from the beginning of the waste retrieval process from this building.

88.8 m^3 of RW of classes D and E (low and intermediate level waste, long-lived) was retrieved from Bld. 157 during 2025 and in total 379.4 m^3 of RW was retrieved from the beginning of the waste retrieval process from Bld. 157.

The SWRF (RU1, RU2/RU3) is operated strictly following the requirements of the Operation Technical Specifications [14, 15]. Safety of the SWRF is ensured by the safe operation of the SSC important to safety and the buildings infrastructure systems, their timely technical maintenance, monitoring, periodic testing and inspections.

All the procedures related to the SWRF operation, RW retrieval and initial processing are carried out in compliance with the requirements stated in the Nuclear Safety Requirements BSR-3.1.2-2017 “Pre-disposal Management of Radioactive Waste at Nuclear Facilities” [16].

1.5 Solid Radioactive Waste Treatment and Storage Facility, SWTSF (B3/4 Project)

The SWTSF consists of the Solid Waste Treatment Facility (SWTF) and the Solid Waste Storage Facility (SWSF).



Figure 1.5-1 SWTSF building

Solid Waste Treatment Facility (SWTF)

The primary function of the SWTF is to treat Group 2 and Group 3 RW. For this purpose, the following operations are performed at the SWTF:

- Reception of the solid RW from the SWRF as well as decommissioning solid waste originating due to dismantling of the INPP equipment;
- Sorting and assessment of the waste according to radiological and physical characteristics;
- Size reduction, where necessary;
- Incineration, where applicable;
- High pressure compaction, where applicable;
- Waste loading into storage containers (optimum capacity);
- Grouting (i.e., filling of containers with grouted waste);
- Waste characterization;
- Waste transfer to the dedicated interim storage facility.

The SWTF consists of two different sorting cells and sorted waste further processing facilities.

In the sorting cells, named after the incoming waste type (G2 and G3 sorting cells, respectively), the waste is processed in parallel streams considering its respective radiological properties. In both sorting cells the waste is sorted by the remotely operated equipment. After the sorting procedure the size reduction and other preparatory actions take place prior to incineration, high force compaction, grouting and / or loading into containers.

Prior to loading the waste into containers, the waste is repeatedly characterized (weight, radionuclide content, dose rate, physical and chemical condition, etc.). Consideration is also given to the waste further technological treatment characteristics, i.e. further interim storage type: in the short-lived waste (SLW) or long-lived waste (LLW) storage facilities.

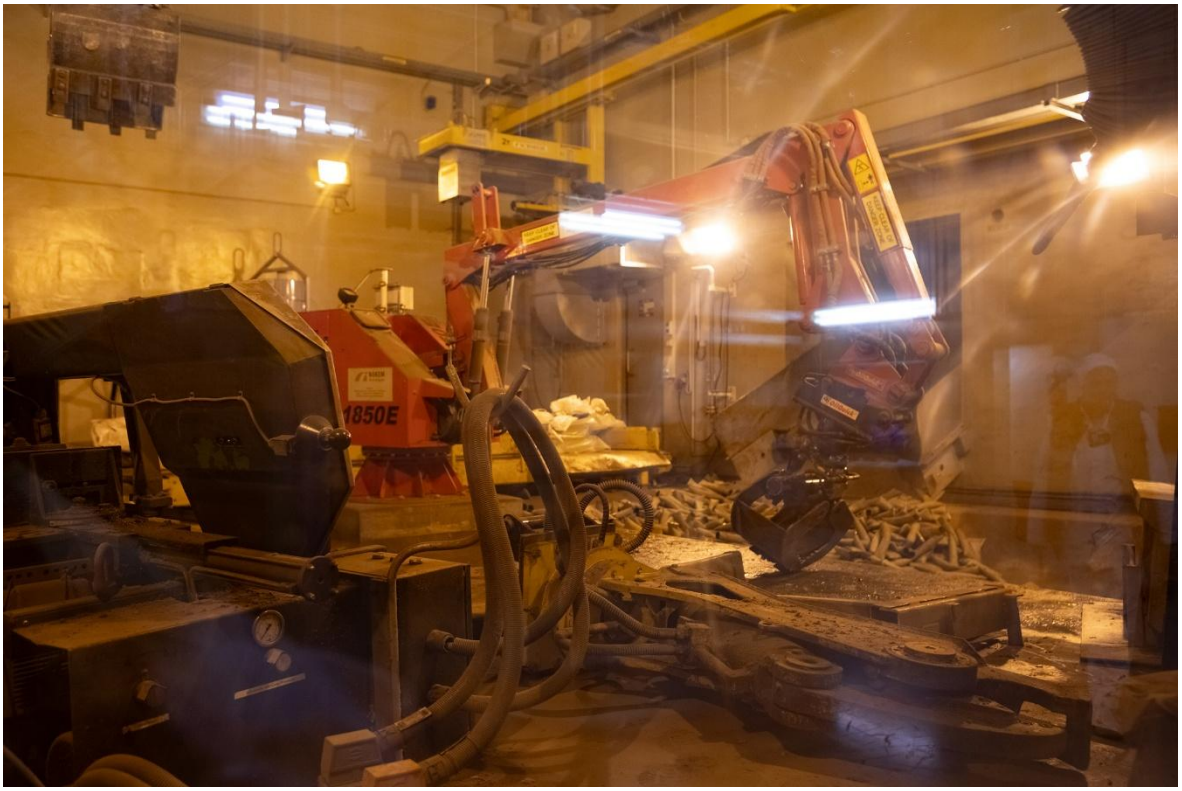


Figure 1.5-2 G2 sorting cell

Solid Waste Storage Facility, SWSF

The SWSF comprises two separate stores directly connected to the SWTF: intermediate short-lived waste storage facility and intermediate long-lived waste storage facility which are directly connected to the SWTF. This ensures transportation of waste containers within the enclosed area.

The SLW waste storage facility can accommodate ~2500 m^3 of treated short-lived waste and allows the waste packages to be stored for a period of 50 years. Waste packages containing classes B and C (low and intermediate level waste, short-lived) waste are stored in the SLW waste storage. The SLW storage is designed so that it can be extended by addition of up to three similar storages (modules), so that the total waste storage volume of 10000 m^3 can be provided.

The LLW waste storage is dedicated for storage of waste packages containing waste of classes D (low level waste, long-lived), E (intermediate level waste, long-lived) and F (spent sealed sources). The LLW storage facility can accommodate ~2000 m^3 of treated long-lived waste and allows the waste packages to be stored for a period of 50 years. The storage facility is provided with separate compartments for spent sealed sources. Besides, the LLW storage

is designed so that it can be extended by addition of additional modules.

Information related to the SWTSF Operation

During 2025 the following solid RW quantities (operational waste) were accepted and treated at the SWTSF:

- Group 1 (class A) combustible waste – 46.1 m^3;
- Group 2 (classes B and C) non-combustible compactable waste –208.7 m^3;
- Group 3 (classes D and E) metal waste – 152.9 m^3;
- Group 3 (class F) spent sealed sources – 0*.

* This group of waste is not treated but only packed into 200 litre drums. After filling up of 4 200-litre drums they are placed into intermediate level long-lived waste (ILW-LL) container and transported for interim storage to the SWSF (Intermediate Level Long-Lived Waste Storage Facility).

The number of formed packages for transfer to the appropriate storage facility is as follows:

- Short-lived waste packages – 187 pieces;
- Long-lived waste (class E) packages – 61 pieces;
- Long-lived waste (class F) packages – 0.

No RW incineration campaigns were conducted during 2025. Combustible waste accumulated during 2025 will be incinerated during the incineration campaigns in 2026.

Acceptance and treatment of the RW at the SWTSF and the SWTSF operation is conducted strictly following the requirements of the Solid Radioactive Waste Treatment and Storage Facility Operation Technical Specification, B3/4 Project [17]. Safety of the SWTSF is ensured by the safe operation of the SSC important to safety and the buildings infrastructure systems, their timely technical maintenance, monitoring, periodic testing and inspections.

All the procedures related to the SWTSF operation, RW treatment and storage is carried out in compliance with the requirements stated in the Nuclear Safety Requirements BSR-3.1.2-2017 “Pre-disposal Management of Radioactive Waste at Nuclear Facilities” [16].

1.6 Landfill Disposal Facility for Short-Lived Very Low-Level Radioactive Waste (B19-2 Project)

The Landfill Disposal Facility consists of two separate facilities:

- 4000 m^3 capacity Buffer Storage Facility built on the INPP site;
- Three Disposal Modules with the capacity of 20000 m^3 of packed RW each. Modules are constructed in sequence, one after another, approximately every 10 years.

The Landfill Disposal Facility is constructed in the vicinity of the INPP, to the south (~50 m) of the ISFSF and the SWTSF sites. The Buffer Storage Facility is built on the INPP territory and is dedicated for accumulation of the waste packages in the quantities necessary for start-up of the waste disposal campaign.

The purpose of the Landfill Disposal Facility is to dispose of very low-level RW (class A) in compliance with the safety and environmental protection requirements providing the necessary protection level of the environment against both the radiological and non-radiological impact.

The Disposal Modules are the above-ground structure, with the where the waste is placed in the standard half height ISO containers, compacted waste plastic bales, and flexible intermediate bulk plastic containers (FIBC package) serving also as barriers protecting against radionuclide discharge into the environment. The waste is loaded in several tiers on the supporting concrete foundation to roughly form a “hill shape” and is fully isolated from the environment by several layers of natural and artificial material layers (engineered barriers) by achieving a smooth, hill-shaped form. Finally, hydro-isolation, drainage and protective layers will be laid on top to prevent water penetration into the waste stack and washout of

radionuclides from the waste to the environment. The filled modules will occupy the area of 4.4 ha.

RW is loaded into the Landfill Disposal Facility with no intention of its retrieval in the future. It is planned to operate the Disposal Modules until the termination of the INPP equipment and buildings dismantling projects. Subsequently the repository will be closed, and the surveillance of the repository will be performed in compliance with the Nuclear Safety Requirements BSR-3.2.2-2016 “Radioactive Waste Repositories” [18]. The active surveillance continues for not less than 30 years and the passive surveillance continues for not less than 70 years. Upon termination of both surveillance periods, monitoring of the Landfill Disposal Facility surrounding environment will still be carried on.



Figure 1.6-1 Formation of protective engineering barriers after waste loading campaign

Landfill Disposal Modules Radioactive Waste Loading Campaigns

After completion of the third campaign of RW packages stacking on the Landfill Disposal Facility site as part of the 1-st Disposal Module “Hot” Trials Programme on 2 May 2025, the Report on completion of the “Hot” Trials Programme was developed and coordinated with the regulatory body on 12 June 2025, as well as the Final Safety Analysis Report of the Landfill Disposal Facility.

On 30 June 2025 the VATESI issued the permit for industrial operation of the Landfill Disposal Facility. The activities related to preparation for the fourth and fifth disposal campaigns are ongoing. The fourth campaign of RW packages stacking started in May 2026 and it is planned to be completed in October 2026.

In total it is planned to conduct 15 RW packages stacking campaigns until fully loading of the Disposal Modules.

The Landfill Disposal facility is operated strictly following the requirements of the Ignalina NPP Short-lived Very Low-Level Radioactive Waste Disposal Facility Operation Technical Specification, B19-2 Project [19]. Inspections and testing of SSC important to safety, their maintenance is carried out in compliance with the INPP Equipment and Facilities Technical Maintenance Regulations [20]. Technical supervision of the Landfill Disposal Modules foundation plate and the structures is carried out at the periodicity and following the requirements and the procedure established in the Instruction on the Technical Supervision of the INPP Buildings and the Territory [21] and B19/2 Facility Sedimentation Monitoring Programme [22].

All the procedures related to the Landfill Disposal Modules operation were carried out in compliance with the requirements stated in the Nuclear Safety Requirements BSR-3.2.2-2016 “Radioactive Waste Repositories” [18].



Figure 1.6-2 Landfill disposal modules site with engineering barriers and protective wall formed after finished waste loading campaign

2 ENVIRONMENTAL MONITORING RESULTS

2.1 Radiological Monitoring Results

Radiological condition of the environmental components and the radiological situation within the Ignalina NPP SPZ, including the ISFSF/SWTSF, Landfill Disposal Facility sites, and the SZ is evaluated based on radionuclide activity concentration measurement results in samples of the environmental components. The environmental components, their sampling periodicity and the sequence are established in the Radiological Environmental Monitoring Programme [8].

Radiological monitoring results, including the meteorological conditions of the INPP region based on the Report of the INPP Region and Maišiagala Radioactive Waste Storage Facility Radiological Monitoring Results for Year 2025 [23] are presented below.

2.1.1 Meteorological Conditions of the INPP Region

Meteorological parameters (temperature, precipitation, relative humidity, atmospheric pressure, wind speed and wind directions (wind rose)) of the INPP region are measured all year-round 24 hours per day since 1987 in compliance with the scope and periodicity established in the Radiological Environmental Monitoring Programme [8]. Meteorological parameters measuring tools are installed at the special meteorological station and on 40 m-height tower located close to Environmental Protection Laboratory (Bld. 438), see Figure 2.1-1 (permanent monitoring point No 1) 6 km to the west from the INPP.

In 2025, the average air temperature was 7.9°C, the coldest month was February with an average monthly air temperature of -4.3°C and the warmest month was July with an average monthly temperature of 18.9°C. The average annual temperature within the observation period (1987-2025) was in the range of 5.0 and 9.0°C.

The average relative humidity value was 83.6% with the minimum value in April (67.6 %) and the maximum value in November (93.4%). Within the observation period (39 years) the average monthly fluctuation of the relative humidity was between 69.8 % and 89.8 %.

During 2025, the mean monthly atmospheric pressure in the INPP region was 995.2 hPa with the minimum value of 987.7 hPa in May and the maximum value of 1008.4 hPa in February.

The minimum average monthly atmospheric pressure fluctuation values from minimum to maximum values were observed during the winter season, the fluctuation range of 23 hPa – 47.5 hPa, more significant fluctuation values were observed during the summer season, the fluctuation range of 15 hPa – 54.6 hPa.

During 2025 the average value of precipitation was 706.4 mm. 52.2% of precipitation was observed during the warm season of the year (April-October) and 47.8% of precipitation was observed during the cold season of the year (November-March).

The average wind speed in 2025 was 3.3 m/s with the maximum speed (gust) of 22.8 m/s registered in October. The wind speed and direction are measured at the height of 40 m. As it could be seen from the Figure 2.1-2 southwestern and southeastern winds prevail.

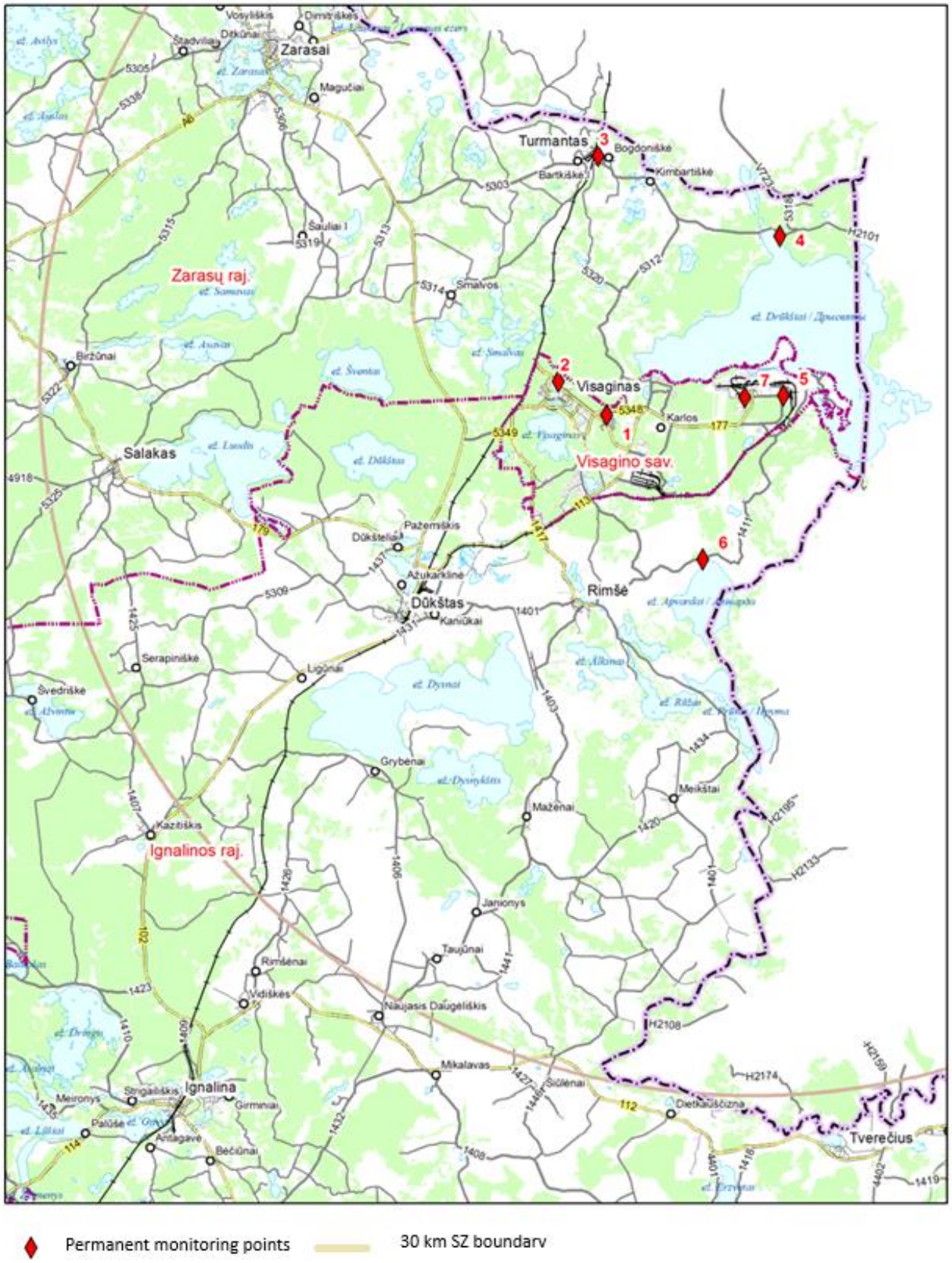


Figure 2.1-1 Layout of permanent monitoring points

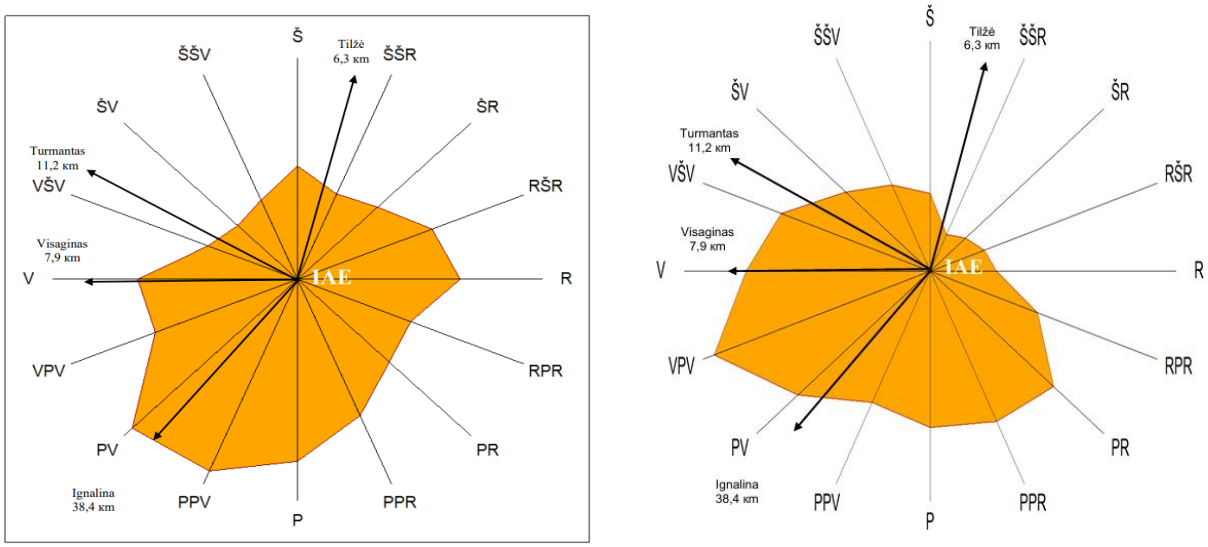


Figure 2.1-2 Wind rose of the INPP region in year 2024 and 2025

2.1.2 Discharge of Radionuclides and Radionuclide Activity Concentration in the Atmospheric Air

The air volume discharged during 2025 into the atmosphere from the INPP organised airborne discharge sources amounted to 1.741E+10 m^3.

Discharges of long-lived radionuclides (H-3 and C-14 excluded) into the atmosphere from the INPP during 2025 amounted to 1.620E+08 Bq which made up 5.59 % from the limit of discharge value of 2.90E+09 Bq/year, as set in the Plan on Radionuclide Releases into the Environment from the Ignalina NPP (hereinafter – Plan) [24]).

H-3 discharges amounted to 2.53E+10 Bq/year (0.25% from the limit discharge value of 1.01E+13 Bq/year, as set in the Plan [24]). C-14 discharges amounted to 7.57E+10 Bq/year (53.30% from the limit of discharge value of 1.42E+11 Bq/year, as set in the Plan [24]). In comparison, in 2024, C-14 discharges amounted to 6.50E+10 Bq/year.

Increase of C-14 discharge in 2025 is due to handling of graphite-containing waste (dismantled equipment from Unit 1 reactor R1 and R2 areas: upper steam-water and lower water communication pipelines and other process communications and their elements, including dismantled fuel channels and reactor control and protection system channels). Exposure dose of the representative person due to C-14 discharges constituted 11.7% of the total exposure dose of the representative person (see Table 2.1-1).

The total activity of radionuclides discharged into the atmosphere during 2025 constituted 1.012E+11 Bq/year and was far below the permissible values set in the Plan [24] (the limit value of 3.81E+13 Bq/year) and constituted 57.6% from the limit of discharge value. Co-60, Sr-90, Cs-137, Nb-94, H-3, C-14 mostly contributed to the activity of the overall discharges from all on-site INPP buildings and facilities.

Table 2.1-1 Radionuclide composition, discharge activity and the resultant dose of the representative person during 2025

Radionuclide	Discharged activity*, Bq/year	Input to dose, %	Dose, Sv
Co-60	6.004E+07	12.94	5.890E-07
Sr-90	4.003E+05	0.08	3.607E-09
Cs-137	1.008E+08	74.84	3.407E-06
Nb-94	8.072E+05	0.35	1.614E-08
H-3	2.528E+10	0.05	2.480E-09
C-14	7.574E+10	11.73	5.340E-07
Total*	**1.012E+11**	**100.00**	**4.552E-06**

* Discharges from the following on-site buildings and facilities contribute to the overall airborne discharges: Bld. 101/1, 101/2, 150, 156, 158/2, 159, 130/2, Landfill buffer storage facility, ISFSF, SWRF, SWTSF.

Based on the exposure dose calculations, the total exposure dose of the representative person resulting from the radionuclide discharges into the atmosphere from the INPP in 2025 constituted 4.552E-03 mSv and it made up 4.55% of the half (0.1 mSv) of the dose constraint - 0.2 mSv.

The table below provides information about the discharged activity from all the new NF and the resultant exposure doses of the representative person.

Table 2.1-2 Activity discharged from all the new NF and resultant doses of the representative person during 2025

Bld.	Discharged activity, Bq/year	Dose, Sv
B1 (ISFSF)*	1.970E+08	1.279E-09
B2 (SWRF)	3.364E+07	3.386E-07
B3/4 (SWTSF)	6.264E+10	2.996E-07

* The air into the environment from the ISFSF building 02 is released via the single discharge source - 30 m ventilation stack from the cask storage room and other ISFSF controlled area premises where casks containing SNF are transferred and handled. Radioactive materials discharged into the atmosphere from the ISFSF building 02 constitute radioactive aerosols, noble gases (Krypton-85).

The analysis indicates that H-3 and C-14 mostly contribute to the total activity of airborne discharges from the INPP NF, whereas the exposure doses determined by Co-60, Cs-137 and C-14 give the major contribution into the total exposure dose of the representative person, see Table 2.1-1.

The major contribution into the total activity discharged from B3/4 facility gives H-3 and C-14, but the input of these radionuclides into the total exposure dose is negligible, if compared to Cs-137. On the other hand, the major contribution into the total activity discharged from B2 facility gives Co-60 and Cs-137. These radionuclides, especially Cs-137, give the most significant input into the total exposure dose, therefore based on the above explanation the estimated exposure dose of the representative person due to discharges from these two facilities is of the same order of magnitude.

2.1.3 Radionuclide Activity Concentration in the Atmospheric Precipitation

Based on the atmospheric precipitation measurement results from 15 sampling points installed within the INPP SZ and SPZ, the natural radionuclide Be-7 mostly determined the radionuclide composition in precipitation in 2025. The average annual radionuclide activity concentration (Be-7 excluded) in the atmospheric precipitation in SZ amounted to 0 Bq/(km^2 day), whereas in the SPZ it amounted to 0.84 10^4 Bq/(km^2 day).

The average concentration of Be-7 in SZ in 2025 was 165 10^4 Bq/(km^2 day) ranging from 42.4 to 483 10^4 Bq/(km^2 day). The average concentration of Be-7 in SPZ was 81.6 10^4 Bq/(km^2 day) ranging from 28 to 189 10^4 Bq/(km^2 day).

The concentration of K-40 in SZ and SPZ was below the detection limit*.

* In this Report the activity value "0" is indicated, if after multiple sampling from the same sampling point the measured radionuclide activity values are below the method's detection limit.

2.1.4 Radionuclide Activity Concentration in the Aquatic Environment

The water volume discharged from the organized waterborne discharge sources during 2025 amounted to 2.053E+06 m^3. The water from all the new NF is not discharged into the environment but is collected into accumulation tanks and then transported to the Liquid Waste Treatment Facility, Bld. 150. The input of wastewater from all the new INPP NF into the overall INPP wastewater volume is negligible. The wastewater volume from all the new NF sites transported to Bld. 150 for processing constitutes only approximately 1% of the total wastewater volume produced as a consequence of activities conducted on the INPP site.

The radionuclide discharges into lake Drūkšiai from the INPP with the wastewater during 2025 (including debalanced water[1]) constituted 5.422E+10 Bq/year (0.36% from the limit value of 1.50E+13 Bq/year specified in the Plan [24]). H-3, Co-60, Cs-137 mostly contributed to the activity of the overall waterborne discharges into lake Drūkšiai from the INPP with the wastewater. The list of discharged radionuclides and their activity values correspond to radionuclides discharged in 2024 (H-3, Co-60, Cs-137).

The total annual tritium activity discharged into lake Drūkšiai through all organized waterborne discharge sources in 2025 amounted to 5.421E+10 and constituted 0.36% of the annual tritium discharge limit (1.50E+13 Bq/year) specified in the Plan [24].

The total annual Cs-137 activity discharged into lake Drūkšiai through all organized waterborne discharge sources in 2025 amounted to 8.480E+08 and constituted 0.28% of the annual Cs-137 discharge limit (3.00E+09 Bq/year) specified in the Plan [24].

The total annual Co-60 activity discharged into lake Drūkšiai through all organized waterborne discharge sources in 2025 amounted to 1.753E+06 and constituted 0.38% of the annual Co-60 discharge limit (4.644E+08 Bq/year) specified in the Plan [24].

[1] Debalansed water is the wastewater originating due to processing of the liquid RW, which is then cleaned at the evaporation installations following the normative requirements and discharged (directed) to the water body. Prior to discharge of the wastewater into the environment sampling and laboratory measurements of the wastewater accumulated in the corresponding tanks are performed to ensure compliance of the wastewater parameters with the normative requirements.

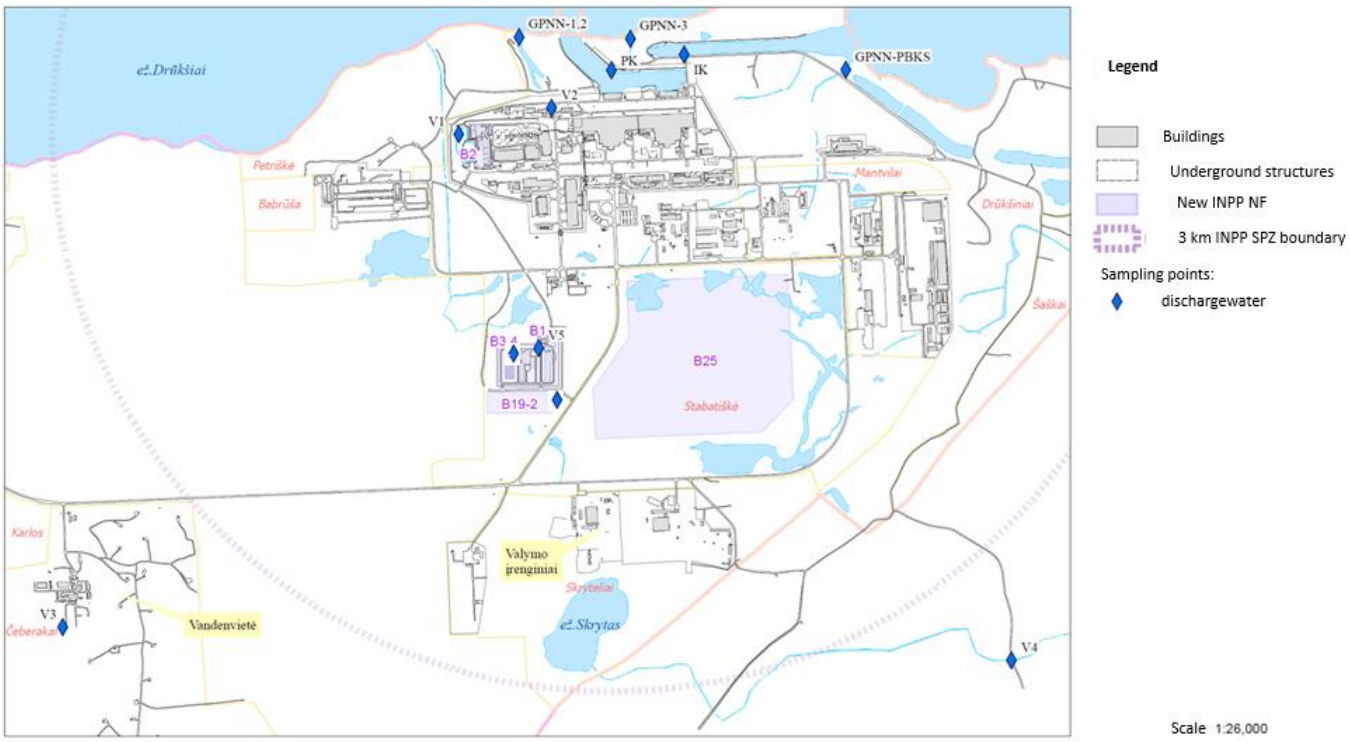


Figure 2.1-3 Layout of water sampling points

IK, PK, GPNN-1,2, GPNN-3, GPNN-SPBKS (Lithuanian abbreviation) denoting the discharge points into Lake Drūkšiai as presented in the order below:

- (IK) Discharge channel – service water discharge from Bld. 101/1,2;
- (PK) Inlet channel;
- (GPNN-1,2) Industrial and Surface Wastewater Drainage channels 1,2 – discharge of rainwater and melted snow from the entire INPP site, discharge of service and debalanced water from Bld. 150 (INPP Liquid Radioactive Waste Treatment and Bituminising Facility);
- (GPNN-3) Industrial and Surface Wastewater Drainage channel 3 – discharges from process equipment located in Bld. 120/1,2 (pumping stations) and drainage water from Bld. 101/1,2;
- (GPNN-PBKS) Dry Type Spent Fuel Storage Facility (the old storage facility) site discharge channel – discharge of rainwater and domestic wastewater from this site.

Sampling points of the water from the accumulation tanks on SWRF, ISFSF, SWTSF, Landfill disposal modules sites before their emptying is also indicated in Figure 2.1-3.

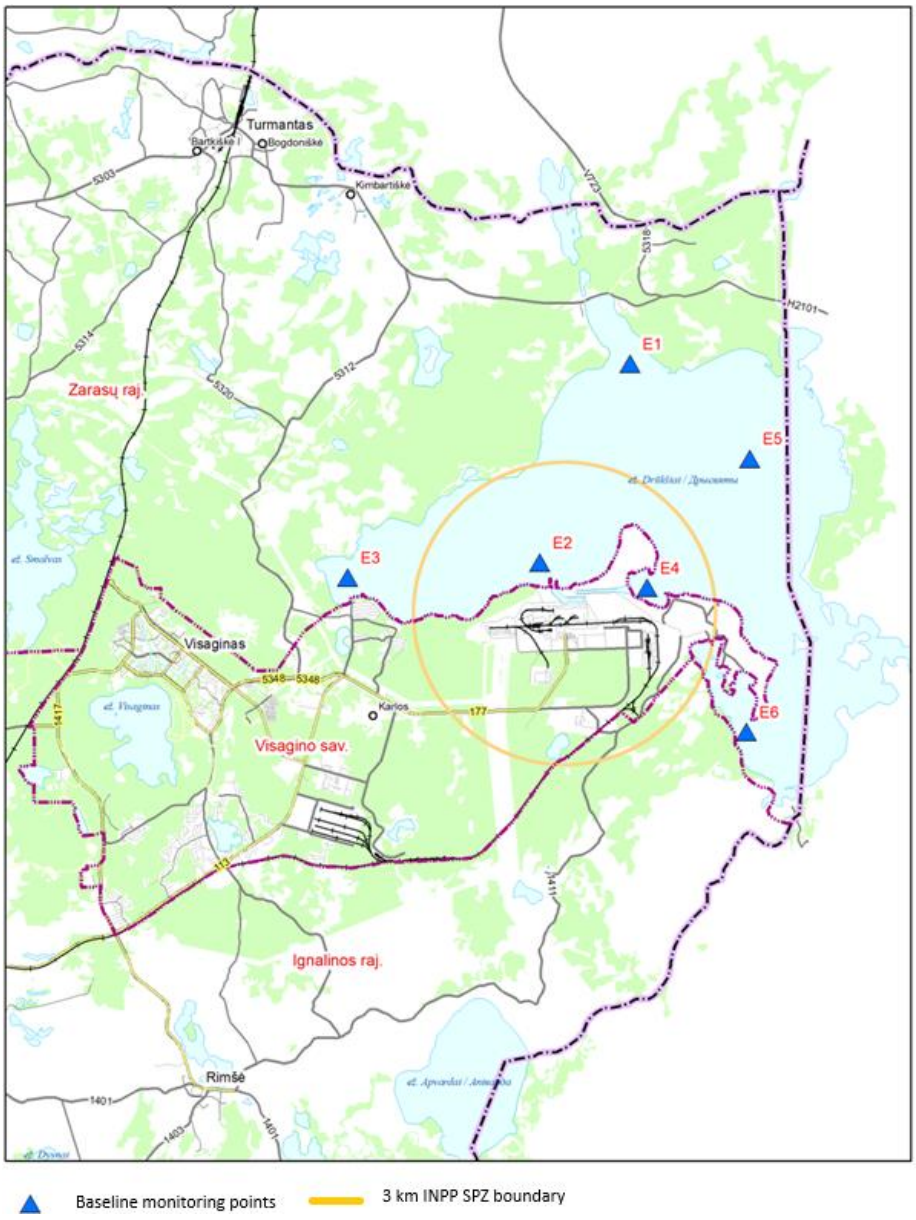


Figure 2.1-4 Baseline sampling locations in lake Drūkšiai

Table 2.1-3 Dose of the representative person due to radionuclide discharges into lake Drūkšiai in 2025

Radionuclide	Discharged activity, Bq/year	Dose, Sv
H-3	5.421E+10	4.40E-08
Co-60	1.753E+06	1.13E-08
Sr-90	0	0
Cs-137	8.480E+06	2.39E-07
Total	**5.422E+10**	**2.95E-07**

The estimated dose of the representative person due to radionuclide discharges into lake Drūkšiai (including debalanced waters) equalled to 2.945E-04 mSv and it made up 0.29 % of the half (0.1 mSv) of the annual dose constraint – 0.2 mSv.

The total estimated dose to the representative person during 2025 due to airborne and waterborne discharges from the INPP equalled to 4.847E-03 mSv, which was 41.3 times less

than the dose constraint of the representative person - 0.2 mSv, i.e. it made up 2.43% of the annual dose constraint.

2.1.5 Radionuclide Activity Concentration in the Water of Monitoring Wells

Within the framework of the Radiological Environmental Monitoring Programme [8] monitoring of the condition of the ground water, its chemical and radiological composition is performed. Periodicity and monitoring scope of the ground water parameters is established in the Radiological Environmental Monitoring Programme [8]. Twice a year, in spring and autumn, the ground water level, gamma nuclide composition, Sr-90 and H-3 are measured.

The layout of monitoring wells on the ISFSF/SWTSF sites and the Landfill Disposal Modules is provided in Fig. 2.1-5 and the layout of monitoring wells on the SWRF site is provided in Fig. 2.1-6.

ISFSF/SWTSF Site

The measurement results of radionuclide activity concentration in the water of monitoring wells on the ISFSF/SWTSF site showed that H-3 concentration in the water of the monitoring wells on the ISFSF/SWTSF site varied between 0 – 11.04 Bq/kg and could be compared to the results of the previous years. The detected values slightly exceed tritium concentration values in natural environment (1÷4 Bq/kg, in non-contaminated surface waters and rainwater [25], but they are significantly lower than the radiological indicator value for drinking water (indicator value – not exceeding 100 Bq/l) set in the Lithuanian Hygiene Standard HN 24:2023 [26], as well as the value of tritium activity concentration of unconditional clearance level, equal to 100 kBq kg^{-1} [27].

Regarding tritium activity concentrations measurements in different reporting periods and by using different measuring counters with different sensitivity it should be said that up to 2024 tritium activity concentration measurements were performed by using older-generation liquid scintillation counter with a minimum detectable activity (MDA) of approximately 8 Bq/kg. In cases where the actual H-3 activity concentration in samples was two or more times lower than MDA, the results were reported as 0 Bq/kg, which means “below the method’s detection limit” rather than complete absence of the radionuclide in the water of monitoring wells. When the new counter with higher efficiency and sensitivity was started to be used at the end of 2023 it made it possible to lower MDA to approximately 5 Bq/kg and, consequently, to detect low concentrations of tritium that were previously below the detection limit. Therefore, the difference between the results is due to instrumental factors, rather than a change in sources or levels of groundwater contamination and does not indicate significant change in the radiological situation but reflects the increased sensitivity of the measuring equipment.

Radiochemical analysis of Sr-90 and gamma spectrometric radionuclide measurements of the water of monitoring wells on the ISFSF/SWTSF site showed that Sr-90 activity concentrations during 2025 remained at the same level as during the previous years and varied within the range of $<3.67 \cdot 10^{-4}$ and $3.68 \cdot 10^{-3}$ Bq/kg and could be compared with the background concentrations.

Co-60 and Cs-137 were not detected in the water of monitoring wells on the ISFSF/SWTSF site.

Landfill Disposal Modules Site

The measurement results of radionuclide activity concentration in the water of monitoring wells on the Landfill Disposal Modules site in 2025 showed that H-3 concentration in the water of the most of monitoring wells on the Landfill Disposal Modules territory varied between 7.50 and 12.79 Bq/kg and could be compared with the previous years activity concentration values with the exception of the monitoring well No 46403 (see Fig. 2.1-5) where H-3 activity concentration was 32.24 Bq/kg in 2025, but which is still more than 3 times lower than the radiological indicator value for drinking water [26].

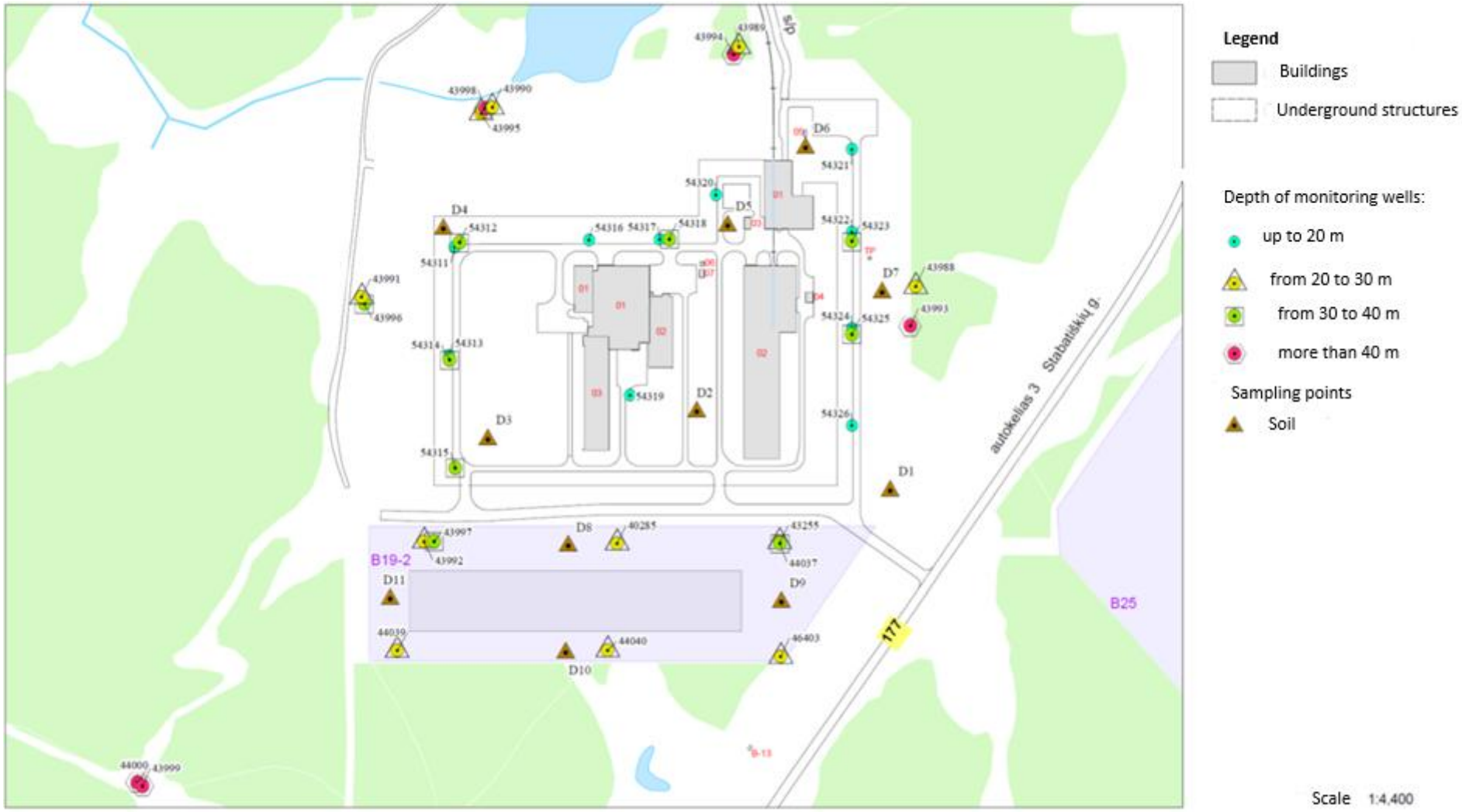


Figure 2.1-5 Layout of monitoring wells within the territory of ISFSF/SWTSF and Landfill Disposal Modules (B19-2)

It should be noted that increase in H-3 activity concentration in monitoring well No. 46403 in 2025 is of localized nature and is not accompanied by simultaneous rise in H-3 activity concentration in other monitoring wells on the Landfill Disposal Modules site. The increased value was recorded during the spring period, but during the repeated sampling in autumn 2025 it indicated clear decrease of H-3 activity concentration to 6.58 Bq/kg, which is comparable with background values. This trend indicates temporary and unstable nature of the increase, likely caused by seasonal hydrogeological factors. Since the monitoring records of the previous years on the Landfill Disposal Modules site and in neighbouring monitoring wells on the Landfill Disposal Modules site demonstrate stability, the most likely causes are temporary hydrodynamic changes in the groundwater regime or redistribution of flows.

Radiochemical analysis of Sr-90 and gamma spectrometric radionuclide measurements of the water of monitoring wells on the Landfill Disposal Modules site showed that Sr-90 activity concentrations during 2025 remained at the same level as during the previous years (preoperational (baseline, zero-background) monitoring results) and could be compared with the background concentrations (the range of Sr-90 values in 2025 was between $<4.79 \cdot 10^{-4}$ and $2.05 \cdot 10^{-3}$ Bq/kg).

Co-60 and Cs-137 were not detected in the water of monitoring wells on the Landfill Disposal Modules site.

SWRF Site

The measurement results of radionuclide activity concentration in the water of monitoring wells on the SWRF site (RU from the existing Interim Radioactive Waste Storage Facilities (Bld. 155, 155/1, 157, 157/1)) showed that H-3 concentrations in most of the monitoring wells varied between 9.74 and 69.55 Bq/kg with the exception of two monitoring wells on the SWRF site (the same monitoring wells as during the previous years) where H-3 concentration values reached 1019.85 Bq/kg (monitoring well No. 29202 located in the vicinity of Bld. 155/1) and 1710.06 Bq/kg (monitoring well No. 29535 located in the vicinity of Bld. 157/1) (see Fig. 2.1-6), but which are still lower than the value of tritium activity concentration of unconditional clearance level, equal to 100 kBq kg^{-1} [27]. It should be noted that the increased tritium activity concentration in the water of the monitoring wells on the Interim Radioactive Waste Storage Facilities (Bld. 155, 155/1, 157, 157/1) site was already detected in 1995 and is related to H-3 sources contained in the legacy waste stored in these buildings and is not related to the operation of the new SWRF. On the contrary, operation of the SWRF, i.e. retrieval and further processing of the stored operational RW, as well as subsequent demolition of these storage facilities (Bld. 155, 155/1, 157, 157/1) will improve the groundwater state.

Tritium activity concentration fluctuation on the SWRF/Interim Radioactive Waste Storage Facilities site is provided in Figure 2.1-7.

Radiochemical analysis of Sr-90 and gamma spectrometric radionuclide measurements of the water of monitoring wells on the SWRF/Interim Radioactive Waste Storage Facilities site showed that Sr-90 activity concentrations during 2025 were in the range of $<7.44 \cdot 10^{-4}$ and $2.43 \cdot 10^{-2}$ Bq/kg and could be compared with the background activity concentrations.

Performed measurements also showed that only very negligible traces of Co-60 were detected just in several monitoring wells (in the vicinity of Bld. 157/1) (ranging between 0.02 and 0.65 Bq/kg) and could be compared with the previous years monitoring results.



Figure 2.1-6 Layout of monitoring wells within the territory of the SWRF/Interim Radioactive Waste Storage Facilities

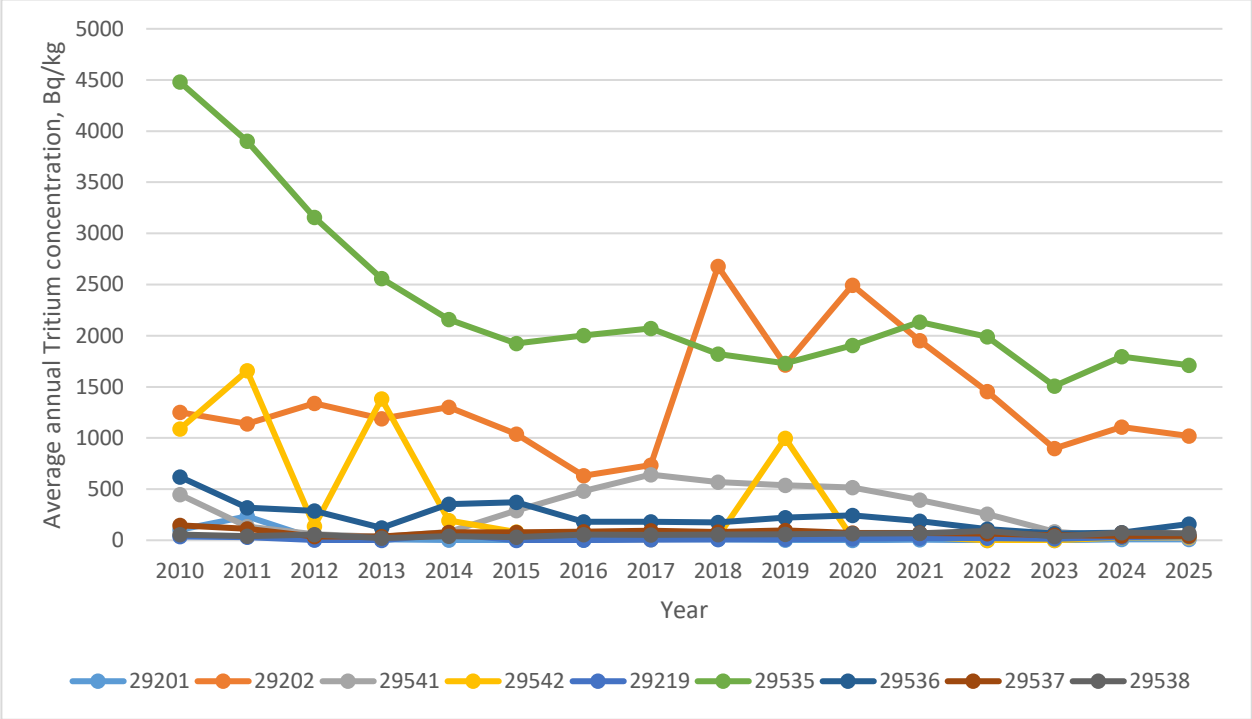


Figure 2.1-7 Tritium concentration in the water of monitoring wells on the SWRF/Interim Radioactive Waste Storage Facilities Site

Based on the multiyear monitoring data, the radionuclide activity concentration in water of the monitoring wells close to the SWRF are decreasing, therefore it could be stated that the physical barriers are preventing the release of radionuclides into the environment. Continuous functioning and control of the INPP buildings drainage systems also serve as preventive measure against contamination of the groundwater. Migration of H-3 towards lake Drūkšiai is not observed since the increased values of H-3 were not detected in the water of the sampling point GPNN-1 (see Fig. 2.1-3) located in close vicinity to the above indicated monitoring well.

In summary, the average annual activity concentration of all measured radionuclides in the water of the monitoring wells is less than the derived radionuclide activity concentrations in drinking water presented in the Lithuanian Hygiene Standard HN 24:2023 [26] except for tritium (taking into consideration that water from the monitoring wells located on the INPP site is not intended for human consumption or any other human needs, as well as the fact that tritium activity concentration transfer by the groundwater from monitoring wells with the increased tritium activity concentration to nearby monitoring wells is not observed). Measured higher values of tritium activity concentration are a consequence of the tritium discharges into the environment and dispersion by groundwater that have occurred during the INPP operation. Tritium is a short-lived beta emitter and its radiological impact at such concentrations is not very significant. Besides, retrieval from the existing Interim Radioactive Waste Storage Facilities of the stored operational RW and subsequent decontamination and demolition of the structures of those storage facilities will eliminate further potential contamination of the groundwater.

To assess the potential consequences of increased tritium concentrations in groundwater, a conservative assessment of the committed effective dose resulting from tritium impact on the human body due to its consumption was performed. According to the Lithuanian Hygiene Standard HN 24:2023 [26] an adult consumes 730 litters of drinking water per year. If the tritium concentration is 1710.06 Bq/kg (one of the highest tritium concentrations in 2025 close to SWRF), then $1.25 \cdot 10^6$ Bq would enter the human body per year. The annual effective dose of the representative person due to tritium impact, assuming that water from the aforementioned

source is the sole source of drinking water, calculated in accordance with Lithuanian Hygiene Standard HN 73:2018 [3] would be 0.023 mSv and is lower than the dose constraint of the annual effective dose - 0.2 mSv set in the Lithuanian Hygiene Standard HN 73:2018 [3]. It should be noted that the water from the monitoring wells located on the INPP site is not intended for human consumption or any other human needs.

2.1.6 Radionuclide Activity Concentration in the Soil

The ISFSF, SWTSF, SWRF and the Landfill Disposal Modules operation in 2025 did not have impact on the radionuclide activity concentration in the soil of the INPP region. K-40, Th-232 and Ra-226 mostly contribute to the activity in soil (natural radionuclides K-40, Th-232 and Ra-226 are not released from the INPP). The radionuclide activity concentration (Ra, Th, K excluded) in the soil of the ISFSF/SWTSF was in the range of 0.93 and 7.35 Bq/kg, whereas radionuclide activity concentration (Ra, Th, K excluded) in sampling points of the SWRF were in the range between 3.49 to 32.9 Bq/kg. Radionuclide activity concentration (Ra, Th, K excluded) in sampling points on the Landfill Disposal Modules area were in the range between 0 to 4.04 Bq/kg. Therefore, K-40, Th-23.2 and Ra-226 and the globally circulating Cs-137[2] mostly contribute to the soil activity.

2.1.7 Radionuclide Activity Concentration in Bottom Sediments of Water Bodies (not related to the ISFSF, SWTSF, SWRF and Landfill Disposal Modules operation)

Natural radionuclides K-40, Th-228 and Ra-226 (not discharged from the INPP) mostly contributed to the activity of bottom sediments of water bodies (the layout of sampling points in water bodies is provided in Fig. 2.1-3 and Fig. 2.1-4). The average radionuclide activity concentration (K-40, Th-228 and Ra-226 excluded) in bottom sediments of water bodies in 2025 was 46.2 Bq/kg. The average Cs-137 activity concentration was 42.9 Bq/kg and the average Sr-90 activity concentration was 5.1 Bq/kg and it could be stated that specific radionuclide activity concentrations during 2025 were within the level of the background values.

2.1.8 Radionuclide Activity Concentration in Plants and Foodstuff

No technogenic radionuclides (Mn-54 and Co-60) were measured in vegetative and animal products sampled during 2025. Small concentrations of globally distributed Cs-137 and Sr-90 (most of which were discharged into the environment after the Chernobyl NPP accident and spread throughout the territory of Lithuania) and the activity of the naturally occurring radionuclide K-40 were measured. The total annual exposure dose due to consumption of measured vegetation- and animal-origin products due to induced radionuclide activity constituted $21.5 \cdot 10^{-4}$ mSv and was practically in line with the previous years results. The calculated exposure dose due to consumption of fish caught in Drūkšiai lake amounted to $2.05 \cdot 10^{-4}$ mSv (K-40 excluded) under condition that 13 kg of fish was consumed per year. The exposure dose due to consumption of mushrooms was $14.9 \cdot 10^{-4}$ mSv (K-40 excluded) under condition that 3 kg of mushrooms was consumed per year.

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[2] Globally circulating radionuclide Cs-137 is one of the major radioactive contamination components of the biosphere, mostly originating due to nuclear weapon tests and nuclear accidents. Cs-137 is the main fission product of the nuclear fission of uranium-235 (reactor fuel). Since Cs-137 concentrations measured in the environmental components in the INPP surroundings are at the background values level, separation of globally circulating Cs-137 from Cs-137 originating due to the INPP activity is not possible. Therefore, the INPP controls radionuclide composition and their activities in all airborne and waterborne discharges into the environment from all the buildings and facilities located in the INPP SPZ, including Cs-137, as well as possesses authorisations for discharge of respective quantities of radionuclides that are coordinated following the procedure established by the legal acts of the Republic of Lithuania. Each year INPP performs environmental impact assessment of the activities conducted by the INPP NF, including assessment of Cs-137 impact.

Measurements showed that natural radionuclides K-40 and Be-7 make the biggest contribution to the total specific activity of algae during 2025. The total specific radionuclide activity concentration (K, Be, Th, Ra excluded) in algae varied in the range of 1.89 and 61.8 Bq/kg.

2.1.9 Dose Rate. Exposure Dose

The INPP environmental monitoring includes monitoring of the exposure dose and dose rate in different locations of the INPP region. Continuous dose rate monitoring is performed by the "Skylink" system stationary gamma detectors. 12 detectors are installed in the INPP SPZ (Fig. 2.1-8). 10 detectors are installed off-site in the 30 km SZ (Fig. 2.1-9). In addition to stationary detectors, dose rate is measured by portable devices 4 times per year in various locations of the SZ.

The measured gamma radiation dose rate within the SZ was between 0.068 and 0.200 μSv/h during 2025. The average gamma radiation dose rate within the SZ was 0.107 μSv/h in 2025.

The measured gamma radiation dose rate within the SPZ was between 0.066 and 0.198 μSv/h during 2025. The average gamma radiation dose rate within the SPZ was 0.109 μSv/h in 2025.

The average gamma radiation dose rate in the INPP region measured by the portable dosimeters was 0.099 μSv/h in 2025. The average dose rate of the representative persons, living in the SZ, due to external radiation background measured by the portable dosimeters was 0.87 mSv/year in 2025.

The gamma radiation dose rate outside the 3 km INPP SPZ boundary does not differ from the gamma radiation dose rate in more distant areas from the INPP.

The radiological impact to the population and the environment due to technogenic radionuclides detected in the atmosphere and water media is far below than the impact due to cosmogenic radionuclides and the globally circulating radionuclide Cs-137.

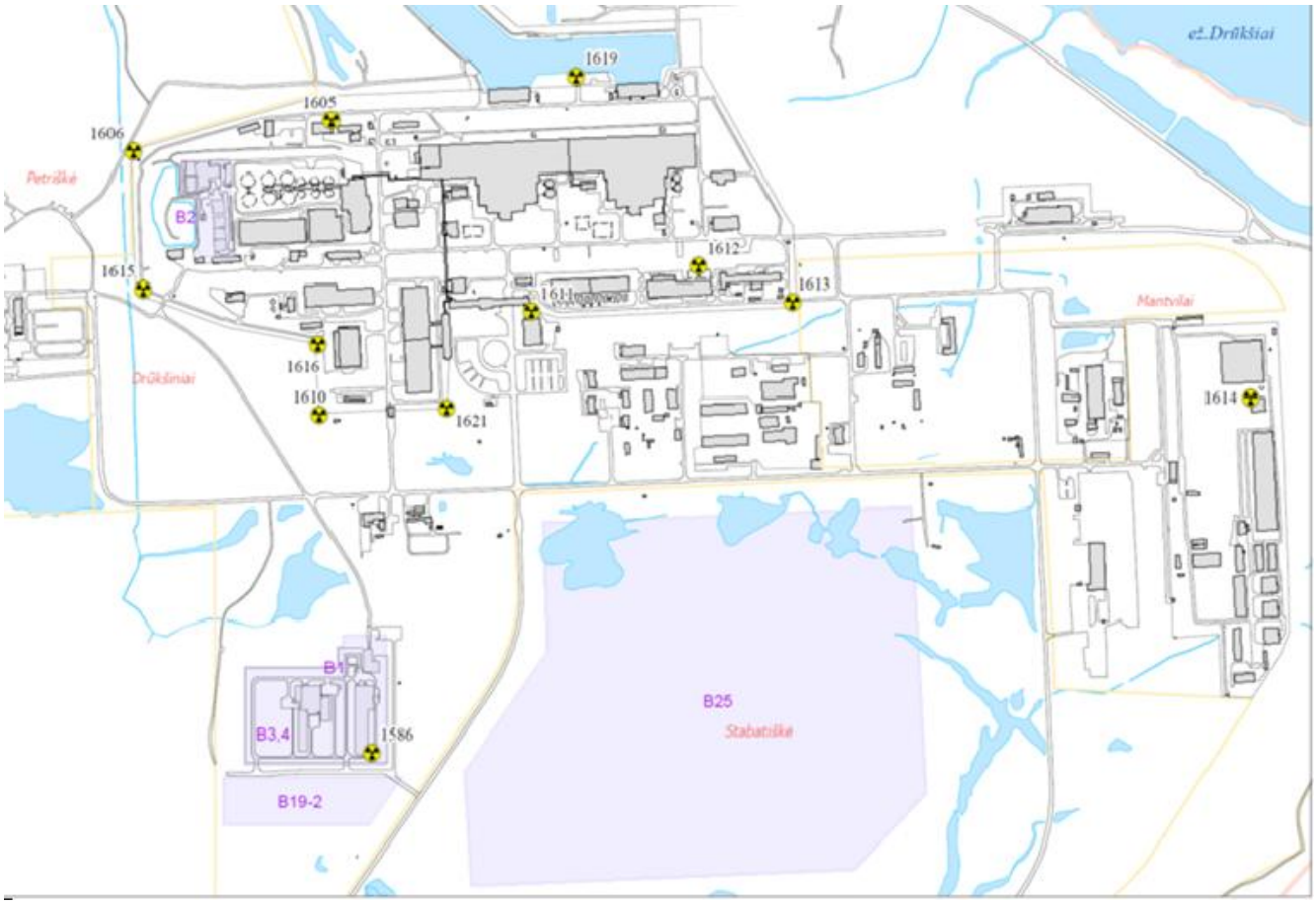


Figure 2.1-8 Layout of the Skylink system gamma detectors (☢) within the INPP 3 km zone

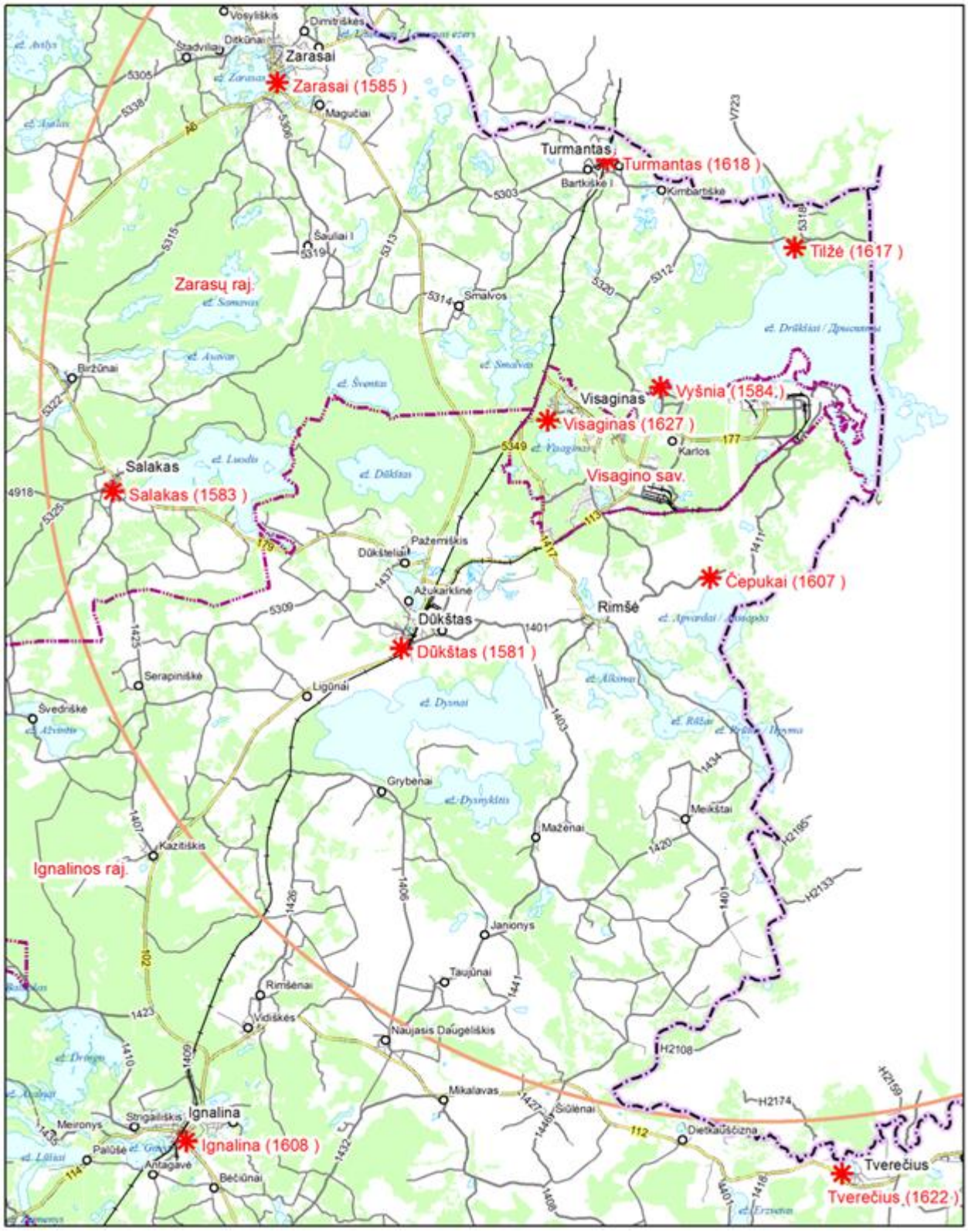


Figure 2.1-9 Layout of the Skylink system gamma detectors (*) within the INPP 30 km zone

The annual ambient radiation dose equivalent of the representative person due to all gamma radiation sources in the SPZ and SZ continuously measured by 24 thermoluminescent dosimeters installed at various distances from the INPP site (Fig. 2.1-10) was in the range of 0.45 mSv and 0.88 mSv with the average value of 0.69 mSv in 2025.

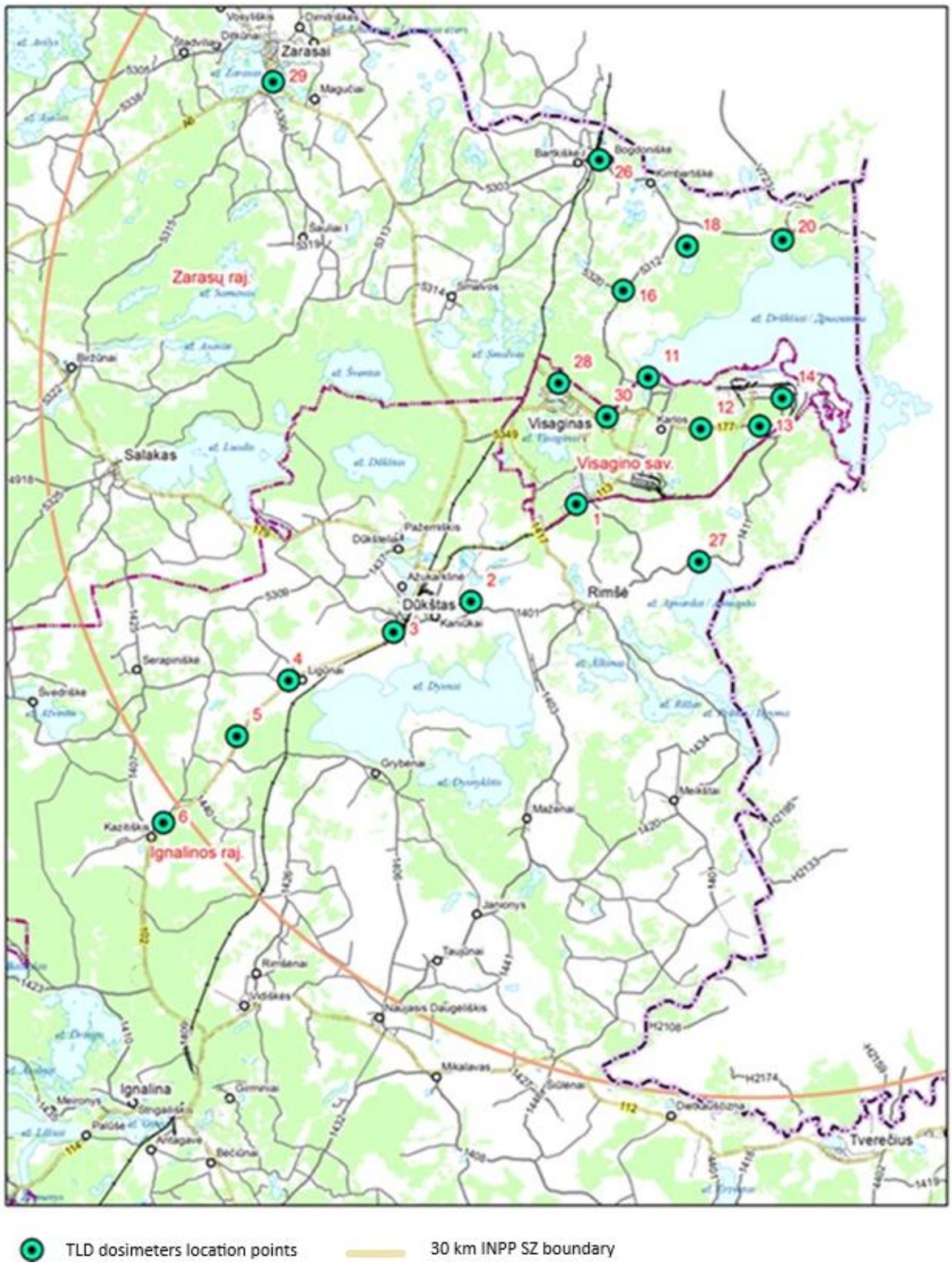


Figure 2.1-10 Layout of TLD dosimeters in the INPP area

The annual ambient dose equivalent measurements on the ISFSF/SWTSF site are performed continuously by 17 thermoluminescent dosimeters (8 dosimeters around the ISFSF and 9 dosimeters around SWTSF) (Fig. 2.1-11). The estimated average annual ambient dose equivalent of the representative person in 2025 based on TLD measurement results was in the range of 0.63 and 0.71 mSv with the average value of 0.66 mSv.

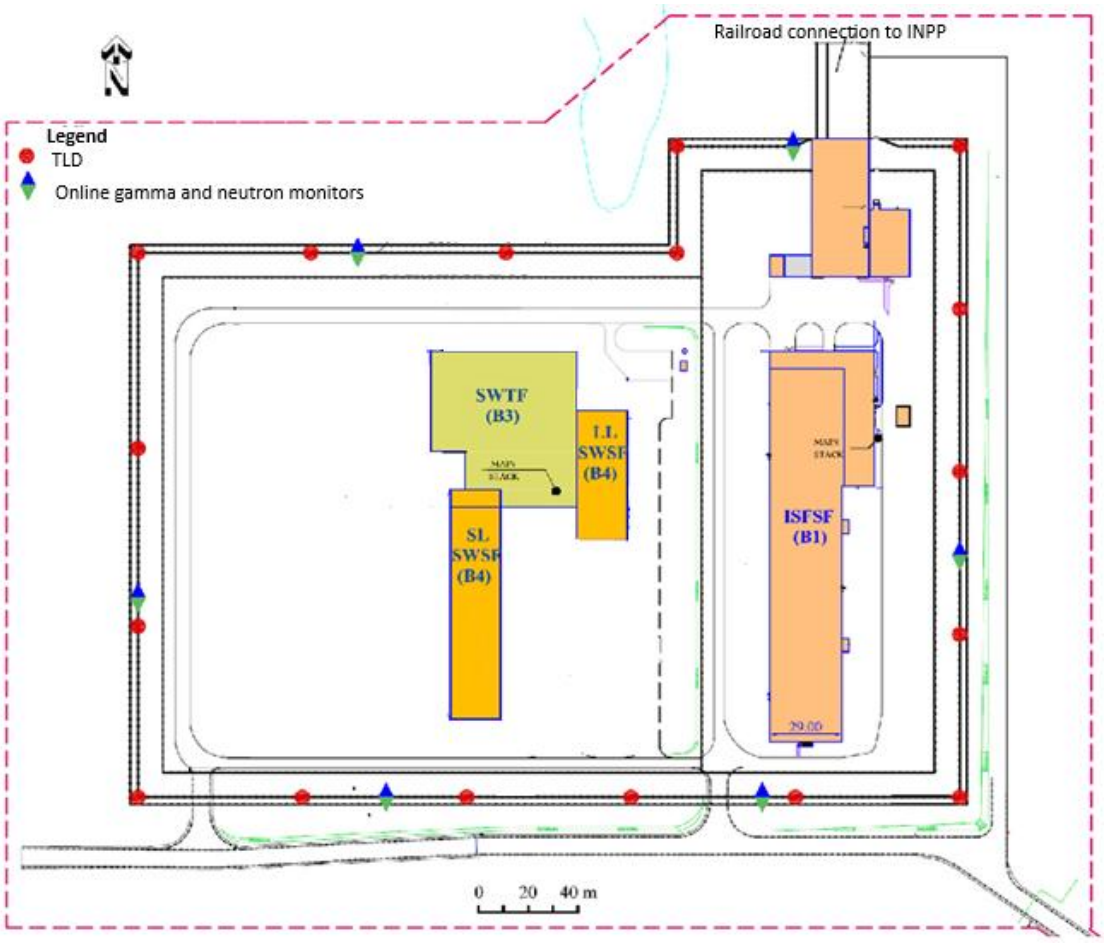
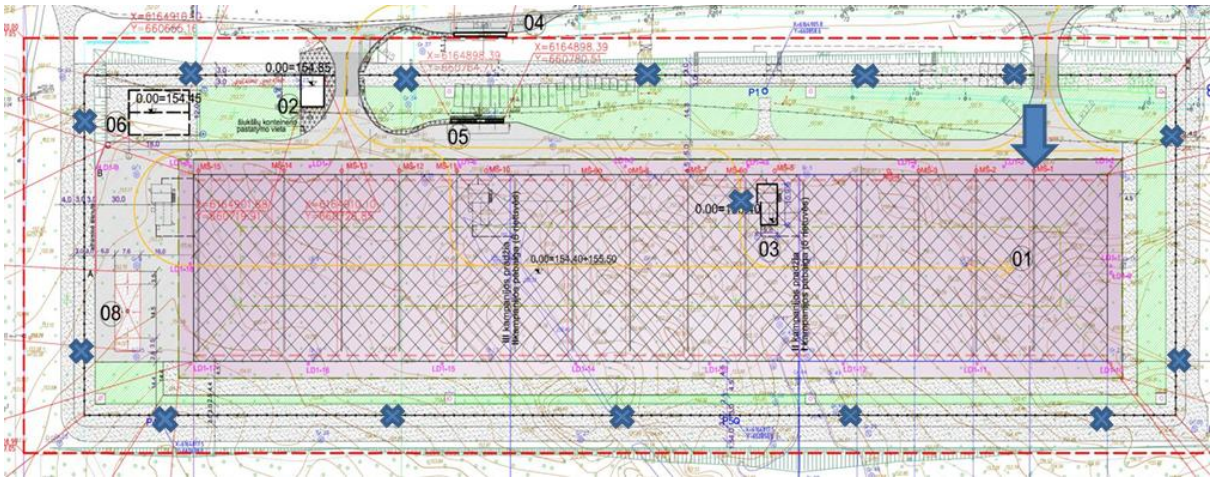


Figure 2.1-11 Layout of TLD, gamma and neutron dosimeters on the ISFSF/SWTSF sites

The annual ambient dose equivalent measurements on the Landfill Disposal Modules site are performed continuously by 14 thermoluminescent dosimeters. The estimated annual ambient dose equivalent of the representative person on the Landfill Disposal Modules site based on TLD measurement results was in the range of 0.62 and 0.65 mSv with the average value of 0.64 mSv.



- TLD location

Figure 2.1-12 Layout of TLD on the Landfill Disposal Modules site

It could be stated that exposure near the INPP SPZ does not differ from exposure in more distant areas. Measurements of the ambient dose equivalent showed that the INPP decommissioning and RW management activities as well as operation of the new NF have no impact on direct exposure of the population of Lithuania.

2.1.10 Conclusions

The operation of the ISFSF, SWRF, SWTSF and the Landfill Disposal Modules, as well as INPP decommissioning activities did not have impact on the radiological situation within the INPP SPZ and SZ in 2025, as:

- No radionuclides that had not been identified in the Plan [24] were identified in airborne discharges from the ISFSF, SWRF, SWTSF and the Landfill Disposal Modules, including no other radionuclide discharge pathways had been identified.
- Airborne radionuclide discharges during the operation of the ISFSF, SWRF and SWTSF in 2025 were 1.969E+08 Bq/year, 3.364E+07 Bq/year and 6.264E+10 Bq/year, respectively. The total annual airborne radionuclide discharges from all the on-site INPP site buildings and facilities amounted to 1.012E+11 Bq/year and were far below the permissible value set in the Plan [24] (57.6% from the limit value of 3.81E+13 Bq/year).
- The estimated dose of the representative person due to radionuclide discharges from the ISFSF, SWRF and SWTSF were 1.279E-09 Sv, 3.386E-07 Sv and 2.996E-07 Sv, respectively and they gave the input of 14.05% into the total exposure dose of the representative person due to airborne discharges from all the buildings and facilities on the INPP site. The total annual exposure dose of the representative person due to airborne discharges from all the on-site INPP buildings and facilities was 4.552E-03 mSv and constituted 4.55 % of the half (0.1 mSv) of the dose constraint - 0.2 mSv [3].
- No uncontrolled waterborne discharges into the environmental waters during the normal operation of all the new nuclear facilities have been detected. The radionuclide discharges into the environmental waters from other buildings and facilities on the INPP site was 5.422E+10 Bq/year (0.36% from the limit value of 1.50E+13 Bq/year). The estimated dose of the representative person due to radionuclide discharges into Lake Drūkšiai (including debalanced waters) was equal to 2.945E-04 mSv and it made up 0.29 % of the half (0.1 mSv) of the dose constraint – 0.2 mSv [3].
- The total estimated dose to the representative person during 2025 due to airborne and waterborne discharges from all the on-site INPP buildings and facilities and all the new NF was equal to 4.847E-03 mSv which was 41.3 times less than the dose constraint of the representative person - 0.2 mSv [3].
- The radiological impact to the population and the environment due to technogenic radionuclides detected in the atmosphere and water media was far below than the impact due to cosmogenic radionuclides and the globally circulating radionuclide Cs-137.
- No operational occurrences breaching the safe operating limits and conditions have occurred. All the required accident elimination instructions and plans related to the ISFSF, SWRF, SWTSF and the Landfill Disposal Modules operation are developed and tested regularly.
- No uncontrolled radionuclide airborne and waterborne discharges from the ISFSF, SWRF, SWTSF and the Landfill Disposal Modules to the environment have been detected.
- The ISFSF, SWRF, SWTSF operation and completion of the Landfill Disposal Modules “hot” tests, including operation of all other on-site INPP buildings and facilities did not have negative impact on the health of the population and the environment of the neighbouring countries.

Besides, based on the measurements conducted by the Radiation Protection Centre of the Republic of Lithuania during 2025 within the scope of the State Environmental Monitoring

Programme 2024–2029 and the summarised results provided in the State Radiological Environmental Monitoring Report for Year 2025 [28], it is concluded that:

- The analysis of obtained radiological environmental monitoring survey and measurements results show that the radiological situation of the environment did not change due to the INPP activities, no higher-than-normal quantities of radioactive substances in the environment were detected;
- The average annual ambient dose equivalents in the Ignalina Region (0.72±0.06 mSv) reliably did not differ from the average annual ambient dose equivalent values in other measuring points within the territory of the Republic of Lithuania. For comparison in the Kupiškis region, with no industrial enterprises and located at a sufficient distance from the INPP, it constituted 0.71±0.06 mSv. Therefore, it indicates that decommissioning activities carried out at the Ignalina NPP do not cause additional exposure of the population of Lithuania;
- Contamination of foodstuffs (meat, fish, milk, grain, vegetables) and drinking water by Cs-137 and Sr-90 was negligible. The analysed radionuclide activity concentrations in samples from the INPP region do not differ from the activity concentrations in samples taken from other regions of Lithuania. The activity concentrations of radionuclides in most of the taken samples is less than 0.1 Bq/kg that leads to negligible exposure of the population;
- It is concluded that the Ignalina NPP decommissioning activities did not have impact on the environmental contamination with radioactive materials. The radiological contamination of samples of foodstuff and drinking water and the external exposure of the population in the Ignalina NPP region did not differ from that in other regions of Lithuania.

The current Lithuanian seismic monitoring network consisting of 4 seismic stations of the Ignalina NPP and 2 very wide-range seismic observation stations connected to the international GEOFON network ensure uniform seismicity monitoring over the entire territory of Lithuania. Though Lithuania is a seismically stable region, but it is still important to conduct seismic observations. No local earthquakes have been registered in the territory of Lithuania. The main focus of Lithuanian seismic monitoring is local seismic events within the Baltic region.

In summary, based on the summarised data presented in the present Report it should be noted that measurements of radionuclide activity discharges into the environment from the INPP site, including all the new NF sites, and assessment of their potential impact on the population showed that in 2025 the exposure of the representative person was local and negligible and constituted a very small proportion of the dose constraint (2.43% of the dose constraint of 0.2 mSv [3]). The radiological impact on the environment and health of the population of more distant neighbouring countries would be even less. If compared to the dose limit applicable for exempted practices - 10 μSv, the exposure dose of the population would constitute 48.5% of this limit.

2.2 Chemical Monitoring Results

The scope of monitoring of the chemical parameters in the INPP environment is established in the INPP Environmental Monitoring Program [9] and the Summarising Report on Monitoring of the Impact of the Facilities located on the SE INPP Territory to the Groundwater for Years 2017-2021 and the Programme for Years 2022-2026 [10].

The summarising results of Lake Drūkšiai water chemical monitoring performed by the accredited INPP Laboratories according to the INPP Environmental Monitoring Program [9] are provided in Table 2.2-1. Sampling locations are provided in Fig. 2.2-1.

According to the data of Table 2.2-1, Lake Drūkšiai water quality indicators comply with the established norms. According to the physical and chemical indicators based on the results of the Environmental Monitoring Report [29] Lake Drūkšiai may be classified as of good ecological state [30].

Table 2.2-1 Pollutants concentration in water of Lake Drūkšiai

Parameters	Limit values according to the established norms	2025		
		Min. value	Average value	Max. value
t, ^{0}C	28	5.7	15.9	22.6
pH,	6 – 9	6.0	7.7	9.0
Suspended solids, mg/l	≤25	<0.9	1.7	8.8
BOD_7, mgO_2/l	≤ 6	0.6	1.2	3.1
COC, mgO_2/l		<30	34.5	54
NH_4-N content, mg/l	≤0.78	<0.032	0.034	0.070
NO_3-N content, mg/l	-	<0.012	0.02	0.081
NO_2-N content, mg/l	≤0.05	<0.006	0.006	0.006
N total content, mg/l	0.90 – 1.20	<1.1	1.20	1.20
PO_4-P content, mg/l	≤0.13	<0.038	<0.038	<0.038
P total content, mg/l	0.051-0.070	0.038	0.060	0.078
Petroleum products content, mg/l	0.2	0.090	0.148	0.240

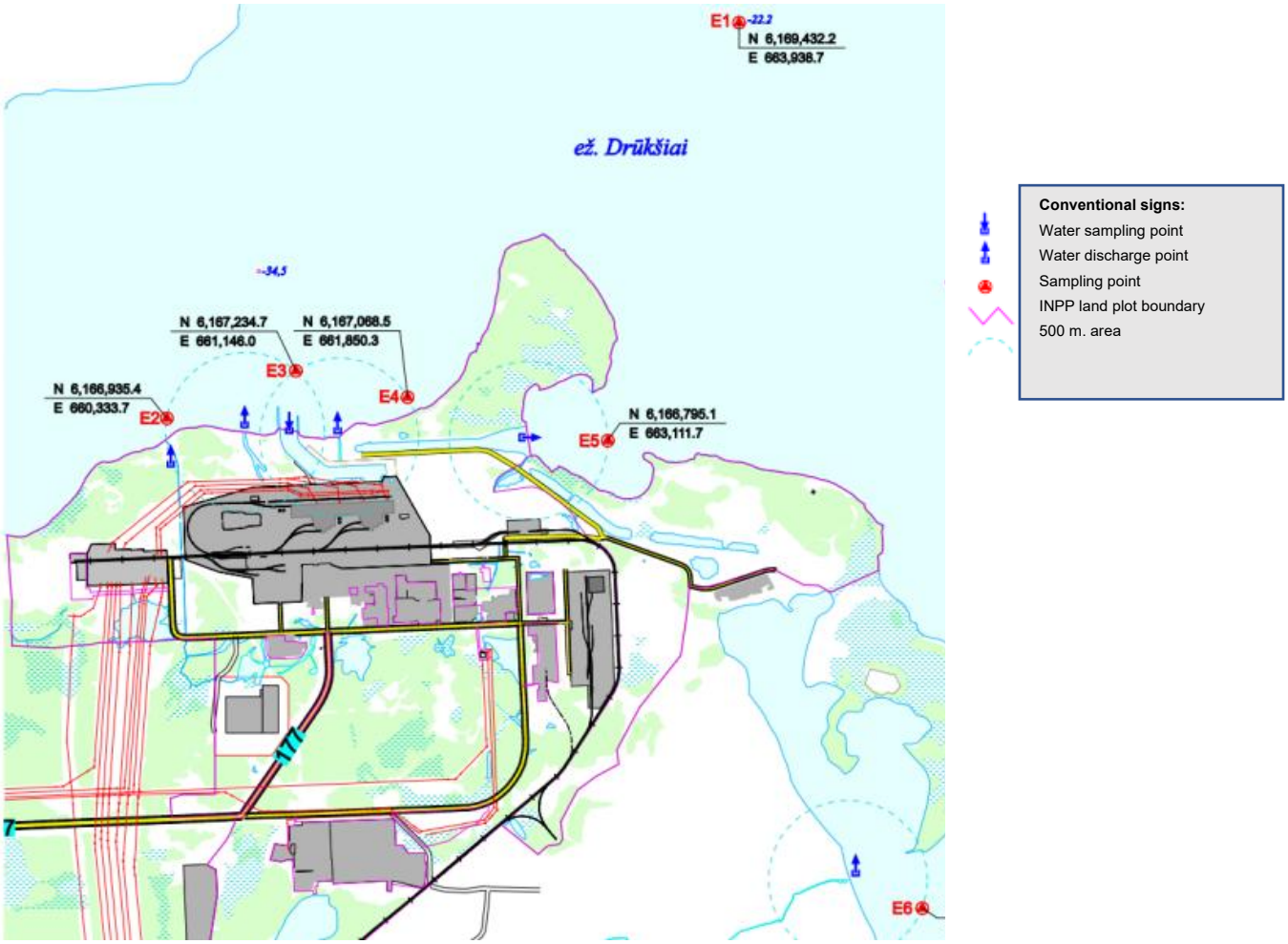


Fig. 2.2-1 Lake Drūkšiai Water Sampling Locations

Regarding the atmospheric air pollution from the established air pollution source (Steam Boiler Plant) during the monitored period (2025), it could be stated that the measured values do not exceed the limit values set in the Permit for Pollution No. TV(2)-3 /TL-U.5-13/2016 [31] and are in line with the discharge values measured in 2024.

In compliance with the Regulation of Environmental Monitoring of Economic Entities [6] analysis of data on monitoring of the impact to the underground water are performed and the related conclusions are submitted each 5 years. The certified INPP laboratory develops simplified annual monitoring reports providing the performed measurement data and the 5-year period aggregated data are summarised in the report developed by the contracted company. The last 5-year period report is developed for the period of 2017-2021. The next 5-year reporting period expires in 2026.

In compliance with the Summarising Report on Monitoring of the Impact of the Facilities located on the SE INPP Territory to the Groundwater for Years 2017-2021 and the Programme for Years 2022-2026 [10] developed on the basis of the requirements of the Regulation of Environmental Monitoring of Economic Entities [6] the following general chemical composition parameters, general pollution indicators and indicators specific to the INPP activity and produced waste are measured 2 times/year: groundwater level, temperature, pH, oxygen concentration, electric conductivity, total chemical analysis (dissolved solids total. hardness total, Permanganate, Bichromate index, Cl, SO_4, NO_2, NO_3, Na, K, Ca, Mg, NH_4), COC, petroleum products total content, heavy metals content, Nitrogen (N), Phosphate (P) total content, PO_4, BOD_7.

In summarising the annual groundwater monitoring results [10], it should be stated that the measured values of the following monitored microelements/heavy metals Cd, Cr, Cu, Hg, Zn did not exceed the maximum permissible concentrations (hereafter referred to as MPC), but the measured values of Pb, Ni in several monitoring wells increased in comparison with the monitoring results of the previous years. Such increase of heavy metals concentrations in the groundwater of the INPP territory stands out against the clean hydrochemical background of the groundwater of the INPP territory according to the other measured parameters and could be explained by the fact that a specific rarely found hydrogeological environment has formed in this area that is favourable for accumulation and migration of these metals [10].

The episodic groundwater pollution by petroleum products is observed but not exceeding the MPC (1 mg/l) with the tendency for reduction. Results of measurement of such parameters as electric conductivity, water temperature, pH, dissolved oxygen show that neither increased salinity of the water, nor abnormal temperatures are observed. In most cases pH of monitoring wells water is within the range of 7-8. However increased acidity (pH<7) of the water is observed in the vicinity of the Solid Radioactive Waste Retrieval Facility and the area between the Units and Lake Drūkšiai. This may be related to technological processes carried out in these facilities using acids. Increased alkalinity (pH>8, pH>10-12) of the water is observed in the area between the Units and Lake Drūkšiai and the Buffer Storage of the Landfill Disposal Facility and is related to degradation of organic materials.

Analysis of Cl, SO_4, NO_2, NO_3, Na, K, Na, Ca, Mg, NH_4-N in samples indicate that only NH4-N is of concern (MPC – 10 mg/l). The maximum values are observed in shallow monitoring wells in the vicinity of the recultivated wastewater sludge dumping site reaching 35.2 – 63.5 mg/l. However, in deeper monitoring wells the values are within the range of background level and do not exceed 0.5 mg/l, thus indicating that only shallow ground water is polluted by sludge.

Permanganate index does not exceed the established value of <20 mg/l O_2 in water of none of the monitoring wells, however BI/COD (MPC<30 mg/l O_2) values within the range of 30 – 100 mg/l O_2 (average pollution level) are detected. The maximum measured values of BOD_7 (MPC 29 mg/l O_2) in the water of monitoring wells close to the recultivated wastewater sludge dumping site (the only monitoring point) reached only 8.4 – 8.5 mg/l O_2 and are considered as negligible.

The total content of N_t (MPC - 30 mg/l) and P_t (MPC - 4 mg/l), as well as PO_4-P (MPC - 3.3 mg/l) are also measured only in the water of monitoring wells close to the recultivated wastewater sludge dumping site and the maximum values of N_t reaches 34.2- 58.3 mg/l. All this nitrogen is bonded into ammonium. The maximum values of P_t and PO_4-P are far below the established MPC and are negligible.

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DETALŪS METADUOMENYS	
Dokumento sudarytojas (-ai)	Lietuvos Respublikos aplinkos ministerija 188602370, A. Jakšto g. 4, LT-01105 Vilnius
Dokumento pavadinimas (antraštė)	REGARDING THE REPORT ON IMPLEMENTATION OF THE POST-PROJECT ANALYSIS PROGRAMME FOR THE NEW INSTALLATIONS OF IGNALINA NUCLEAR POWER PLANT FOR THE YEAR 2025
Dokumento registracijos data ir numeris	2026-10-13 Nr. D8(E)-4087
Dokumento gavimo data ir dokumento gavimo registracijos numeris	–
Dokumento specifikacijos identifikavimo žymuo	ADOC-V1.0
Parašo paskirtis	Pasirašymas
Parašą sukūrusio asmens vardas, pavardė ir pareigos	Giedrė Ričkutė, Viceministras
Sertifikatas išduotas	GIEDRĖ RIČKUTĖ, Lietuvos Respublikos aplinkos ministerija LT
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